



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19406-1415

December 8, 2009

Mr. John Carlin
Vice President
R. E. Ginna Nuclear Power Plant, LLC
1503 Lake Road
Ontario, NY 14519

**SUBJECT: R. E. GINNA NUCLEAR POWER PLANT - NRC POST-APPROVAL SITE
INSPECTION FOR LICENSE RENEWAL INSPECTION REPORT
05000244/2009009**

Dear Mr. Carlin:

On November 4, 2009, the U. S. Nuclear Regulatory Commission (NRC) completed a Region I inspection of your post-approval license renewal activities at the R. E. Ginna Nuclear Power Station. The enclosed report documents the results of the inspection, which were discussed during an exit meeting on November 4, with Jay Wells and other members of your staff.

The purpose of this inspection was to examine the activities under your renewed operating license related to the completion of commitments made during the renewed license application process and compliance with the conditions of your license. Under the renewed operating license, entry into the period of extended operations occurred on September 19, 2009. The inspection followed up on areas identified in Inspection Report 05000244/2009007, dated July 2, 2009. Within these areas, the inspection involved examination of selected procedures and representative records, observations of activities, and interviews with personnel. On the basis of the samples selected for review, the inspection determined that the license renewal commitments at Ginna were met.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web-site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

A handwritten signature in black ink, appearing to read "Richard Conte", is written over the typed name.

Richard Conte, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No. 50-244
License No. DPR-18

Mr. John Carlin
Vice President
R. E. Ginna Nuclear Power Plant, LLC
1503 Lake Road
Ontario, NY 14519

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Sincerely,

/RA/

Richard Conte, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No. 50-244
License No. DPR-18

SUNSI Review Complete: RJC (Reviewer's Initials)

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| NAME | GMeyer/GWM | GDentel/GTD | DRoberts/DJR | RConte/RJC | |
| DATE | 11/30/09 | 12/02/09 | 12/07/09 | 12/03/09 | |

OFFICIAL RECORD COPY

J. Carlin

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Enclosure: Inspection Report No. 05000244/2009009
w/Attachments

cc w/encl: Distribution via ListServ

Distribution w/encl: (via E-mail)

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U. S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No.: 50-244

License No.: DPR-18

Report No.: 05000244/2009009

Licensee: R. E. Ginna Nuclear Power Plant, LLC (Ginna)

Facility: R. E. Ginna Nuclear Power Plant

Location: Ontario, New York

Dates: November 2 - 4, 2009

Inspectors: Glenn Meyer, Senior Reactor Inspector

Approved by: Richard J. Conte, Chief
Engineering Branch 1
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000244/2009009; 11/02/2009 – 11/04/2009; R. E. Ginna Nuclear Power Plant; License Renewal Commitments Inspection.

The report covers an inspection of the implementation of license renewal commitments, which was performed in accordance with NRC Manual Chapter 2516 and NRC Inspection Procedure 71003 by a region-based engineering inspector. The inspector reviewed the remaining uncompleted areas of the One-Time Inspection Program, which had been identified during the NRC team inspection of license renewal commitments in May 2009, prior to the period of extended operations. The inspector concluded that commitments associated with the issuance of the renewed operating license were met at the R. E. Ginna Nuclear Power Plant.

A. NRC-identified and Self-Revealing Findings

None

B. Licensee-Identified Violations

None

REPORT DETAILS

4. OTHER ACTIVITIES (OA)

4OA5 Post-Approval Site Inspection for License Renewal

On May 19, 2004, NRC issued a renewed operating license for the R. E. Ginna Nuclear Power Plant, based on review of the Ginna License Renewal Application (LRA) submitted on August 1, 2002. The "Safety Evaluation Report (SER) Related to the License Renewal of R. E. Ginna Nuclear Power Plant," (NUREG-1786), issued in May 2004, contained Appendix A, Commitment Listing, which listed 40 commitments. Ginna made the commitments to provide aging management programs to manage aging effects on structures, systems, and components (SSCs) prior to and during the period of extended operation, as well as other information. The period of extended operation began on September 19, 2009, upon expiration of the original license term.

Inspection Procedure (IP) 71003, Post-Approval Site Inspection for License Renewal, was completed in three phases. In Inspection Report 05000244/2008008, dated June 13, 2008, inspectors reviewed license renewal activities and commitments during the refueling outage prior to the period of extended operation. In Inspection Report 05000244/2009007, dated July 13, 2009, an inspection team reviewed the remaining license renewal activities and commitments prior to the period of extended operation. In both inspections the inspectors did not identify any instances of incomplete commitments with respect to timeliness or adequacy. Nonetheless, the second inspection noted that the remaining uncompleted areas of the One-Time Inspection Program merited further NRC followup during the period of extended operation.

a. Inspection Scope (IP 71003)

In accordance with Inspection Procedure (IP) 71003, the inspector reviewed supporting documents including completed tests and inspections, performed interviews, and performed visual inspections of structures and components. The inspection addressed One-Time Inspection Program activities subsequent to Inspection Report 05000244/2009007.

b.1 One-Time Inspection Program

The One-Time Inspection Program addressed the inspections to verify current aging management programs were effective regarding corrosion and selective leaching of materials. The program involved 30 groups of materials and environments, and for each group involved an Inspection Scope Report (ISR) for inspection sample determinations and a Final One-Time Inspection Report (FOTR) for summarizing inspection results. The program involved 236 non-destructive examination (NDE) inspections in the first round, composed of a combination of ultrasonic inspections of pipe wall thickness and visual inspections of other components. The inspector reviewed the program results summary dated October 29, 2009.

Enclosure

In three material/environment groups, i.e., cast iron in drainage raw water, carbon steel in raw water, and carbon steel in treated water, sufficient corrosion was found that merited periodic inspection of all components in the group. Ginna revised program documents to perform these inspections within the Periodic Surveillance and Preventive Maintenance (PSPM) Program. The inspector reviewed all inspection reports, ISRs, and FOTRs for the applicable groups, related corrective action program condition reports (CRs), and program changes to accomplish the revised aging management approach.

In five additional material/environment groups, some inspections had not met the acceptance criteria, which necessitated further evaluation, and in some cases, expansion of the inspection samples. The inspector reviewed all inspection reports, ISRs, and FOTRs for the applicable groups, and related CRs. The inspector noted that Ginna had completed the evaluations and had determined appropriate followup actions, which generally involved followup inspections in three years. Within the five groups, a total of 39 additional inspections were specified. The inspector observed copper alloy sump discharge piping in emergency diesel generator (EDG) rooms A and B, which was scheduled for followup inspection.

For the material/environment group of cast iron in raw water, six components (bearing housing/coolers) in the auxiliary feedwater and safety injection systems were largely inaccessible for inspection and the remote visual inspections found some corrosion on the coolers themselves. Further, destructive evaluation of a cooler access port identified some selective leaching (removal of iron elements from cast iron structure). The inspector reviewed all inspection reports, ISR, and FOTR for this group and related CRs. Given the likelihood of continued degradation of these components, Ginna determined that the components should be replaced, and plans were underway for the procurement and maintenance planning to accomplish this plan.

Lastly, the inspector reviewed four additional material/environment groups in which inspections had been scheduled during the prior NRC inspection, but the NDE inspections were completed subsequent to the NRC inspection. The inspector reviewed all inspection reports, ISRs, and FOTRs for these groups and related CRs. Also, the inspector observed a neoprene ventilation component in the standby AFW room, which was scheduled for replacement.

Overall, the inspector determined that Ginna had performed the initial round of inspections required by the One-Time Inspection Program, Ginna had performed adequate evaluations of all inspections, and the followup inspections, aging management program actions, and component replacements were appropriately completed or scheduled to be completed.

b.2 Commitment Change Process

Inspection Report 05000244/2009007, dated July 13, 2009, noted the following:

"[T]esting for one material/environment group had been completed acceptably but was not evaluated, because the licensee determined that NRC guidance did not mandate testing on this material/environment group. The inspectors noted that both approaches [complete evaluation or remove from program] were technically acceptable, but deletion of testing discussed in the License Renewal Application should have involved use of commitment change process, which had not been done."

The inspector reviewed Commitment Change Evaluation 2009-003, which had been approved on October 16, 2009, and had removed the material/environment group from the One-Time Inspection Program. The inspector noted that at the time of the previous inspection, this issue was in progress of being evaluated. The inspector determined that the program change had been accomplished in an appropriate manner.

c. Overall Conclusions

The inspector determined that One-Time Inspection Program activities were appropriately completed. The inspector did not identify any substantive instances of incomplete license renewal commitments with respect to timeliness or adequacy. In combination with other post-approval license renewal inspections, the inspector concluded that commitments associated with the issuance of the renewed operating license had been met at the R. E. Ginna Nuclear Power Plant.

O4 MANAGEMENT MEETINGS

Exit Meeting Summary

The inspectors presented the results of the inspection to Mr. Thomas Paglia and Mr. Jay Wells and other members of Ginna's staff on November 4, 2009. The inspectors confirmed that no proprietary material was examined during the inspection.

ATTACHMENT: SUPPLEMENTAL INFORMATION

Enclosure

ATTACHMENT

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee Personnel

| | |
|------------|---|
| T. Paglia | Manager, Integrated Work Management |
| J. Wells | General Supervisor, Engineering Programs |
| M. Fallin | Principal Engineer, Constellation License Renewal |
| T. Harding | Director, Licensing |
| K. Kemp | Aging Management Coordinator |
| M. Ruby | Principal Engineer, Licensing |

Observers

| | |
|---------------|---------------------------------|
| R. Saunderson | Nine Mile Point license renewal |
|---------------|---------------------------------|

LIST OF DOCUMENTS REVIEWED

Miscellaneous Documents:

LR-OTI-PROGPLAN, One-Time Inspection Program, Rev 5

One-Time Inspection Program Summary, October 29, 2009

Calculation DA-ME-09-015, May 8, 2009

Calculation - MSSV & ARV Discharge Stacks Piping Minimum Wall Thickness

Action Item NL-2009-000025 with actions -001 through -007

Commitment Change Evaluation No. 2009-003 approved Oct. 16, 2009

Aux Feed Pump Water Bearing Cooler Material Analysis, August 5, 2008

Stainless Steel Swagelok Inspection, November 3, 2008

Inspection Scope Reports (ISRs) (all NL-2007-000081-):

005, Sept. 3, 2009

009, Sept. 5, 2009

010, Oct. 28, 2009

011, April 25, 2009

016, Oct. 10, 2009

017, Oct. 28, 2009

018, Aug. 23, 2009

019, Oct. 29, 2009

021, Oct. 29, 2009
022, Aug. 23, 2009
025, Oct. 9, 2009
026, Aug. 23, 2009
027, Oct. 27, 2009

Final OTI Reports (FOTRs) (all NL-2007-000081-):

005, Sept. 5, 2009
009, Sept. 9, 2009
010, Oct. 28, 2009
011, Oct. 29, 2009
016, Oct. 29, 2009
017, Oct. 28, 2009
018, Aug. 23, 2009
019, Oct. 29, 2009
021, Oct. 29, 2009
022, Aug. 23, 2009
025, Oct. 9, 2009
026, Aug. 23, 2009
027, Oct. 27, 2009

CRs: All 2009-

000441
000493
000824
001061
001393
001638
002928
003360
003473
003488
003782
004157
005588
005948
006083

NDE Inspections:

BOP-PT-09-032

BOP-UT-08-064
BOP-UT-08-065
BOP-UT-08-066
BOP-UT-08-067
BOP-UT-08-068
BOP-UT-08-069
BOP-UT-08-070

BOP-UT-08-071
BOP-UT-08-072

BOP-UT-09-045
BOP-UT-09-047
BOP-UT-09-049
BOP-UT-09-108

BOP-VT-08-080
BOP-VT-08-081
BOP-VT-08-228
BOP-VT-08-229
BOP-VT-08-409
BOP-VT-08-410
BOP-VT-08-458

BOP-VT-09-240
BOP-VT-09-246
BOP-VT-09-273
BOP-VT-09-351
BOP-VT-09-438
BOP-VT-09-587

Eddy Current Inspection Report of Feedwater Heater 5B, October 20, 2006
Eddy Current Inspection Report of Feedwater Heater 3A, April 26, 2008

LIST OF ACRONYMS

| | |
|------|--|
| AFW | Auxiliary Feedwater |
| AMP | Aging Management Program |
| CR | Condition Report |
| EDG | Emergency Diesel Generator |
| LRA | License Renewal Application |
| NDE | Non-Destructive Examination |
| NRC | Nuclear Regulatory Commission |
| PEO | Period of Extended Operation |
| PSPM | Periodic Surveillance and Preventive Maintenance |
| SSC | Structure, System, and Component |